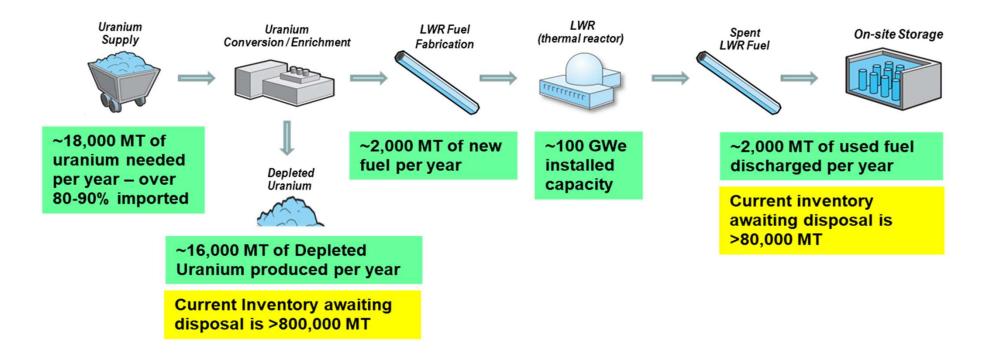
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# **Nuclear Fuel Cycle**



### Current U.S. (Open) Fuel Cycle



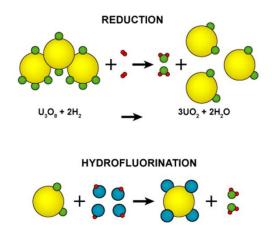
# **Mining and Milling**

- A reliable source of enriched uranium is needed for reactor demonstrations and deployment
- Uranium is mined in three ways:
  - In situ leaching
  - Open pit mining
  - Underground mining
- Wyoming is the US's largest uranium producer, but most of our uranium is imported.
- Advanced reactors utilize low enriched uranium
  - Most use 19.75% U-235 enriched uranium (High-Assay Low Enrichment Uranium)
  - This is in comparison to LWRs which currently use < 5 %</li>
- Supply chains for uranium and enrichment will be needed to support future reactor deployment.



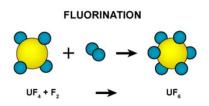
### Conversion

- Natural uranium (U) contains two isotopes
  - 0.7% is "fissile" <sup>235</sup>U which is easily split in a reactor
  - 99.3% is "fertile" <sup>238</sup>U which is not
- Natural uranium is in an oxide form (U<sub>3</sub>O<sub>8</sub>)
  - At the conversion plant U<sub>3</sub>O<sub>8</sub> is converted to uranium hexafluoride (UF<sub>6)</sub>, which is a solid at room temperature but a gas at slightly higher temperatures
  - UF<sub>6</sub> is stored and shipped in large cylinders
- Fresh nuclear fuel in current reactors is ~4.5% fissile content, requiring enrichment:
  - 1. Convert the Uranium to a gas
  - 2. Spin the gas at high speeds in centrifuges
  - 3. The lighter <sup>235</sup>U partially separates from the heavier <sup>238</sup>U

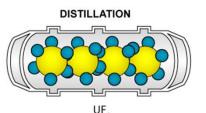




Green salt (UF<sub>4</sub>) is formed when uranium dioxide (UO<sub>2</sub>) is reacted with anhydrous hydroflouric acid (HF)



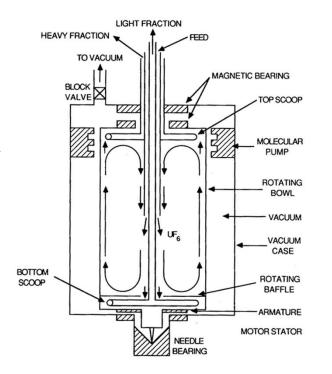
Green salt  $(UF_4)$  is contacted with fluorine gas  $(F_2)$  to form uranium hexaflouride  $(UF_6)$  gas

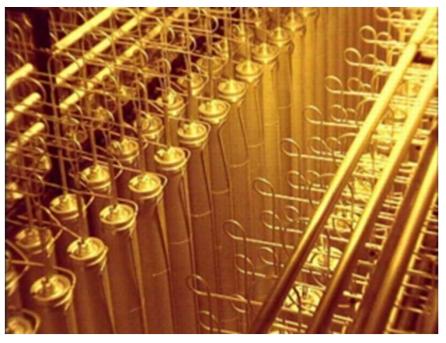


http://www.theupa.org/uranium\_technology/conversion/

### Enrichment

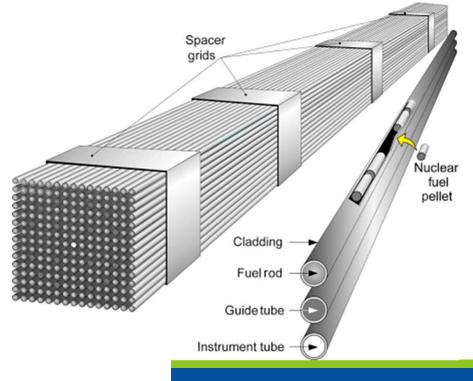
- Enrichment involves passing UF<sub>6</sub> through high-speed centrifuges
  - In each centrifuge the ratio of <sub>235</sub>U to <sub>238</sub>U is changed slightly into a "heavy fraction" scooped from the outside and a "light fraction" scooped from the inside
  - Hundreds of centrifuges are linked in "cascades" to produce enriched U while also generating larger amounts of depleted U that is discarded





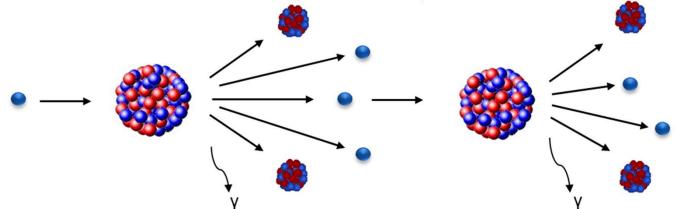
### **Fuel Fabrication**

- Fuel fabrication is a multi-step process
  - $UF_6$  is received and converted to an oxide powder (UO<sub>2</sub>)
  - The powder is pressed into pellets
  - The pellets are heated (sintered) to create a ceramic
  - The ceramic pellets are stacked inside cladding to make fuel rods, which are welded shut
  - The fuel rods are loaded into assemblies which are typically 16 to 17 inches square and ~16 feel long
  - The very slightly radioactive assemblies are inspected, then shipped to reactors
  - 150 to 250 assemblies are loaded into a core, depending on the reactor size.





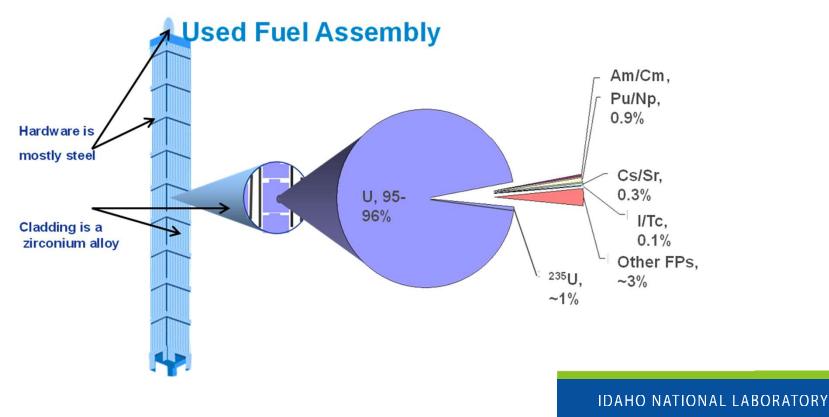
- In the reactor, nuclear fuel is irradiated for ~4.5 years
  - During this time, <sup>235</sup>U is fissioned (split) by neutrons to produce electricity
  - Fissioning results in heat, fission products (smaller atoms) and more neutrons to continue the process



- After the fissile material is depleted and fuel is "spent" and replaced during refueling
  - The highly radioactive used fuel is initially stored in water to "cool", then may be transferred to dry storage

### **Used fuel characteristics (1/3)**

- Used fuel is composed mostly of <sup>238</sup>U, along with:
  - ~4% fission products
  - ~1% residual <sup>235</sup>U
  - ~1% heavier "transuranic" isotopes from neutron capture of <sup>238</sup>U (breeding)



#### **Used fuel characteristics (2/3)**

<sup>1</sup> H																	<sup>2</sup> He
<sup>3</sup> Li	<sup>4</sup> Be											5 B	<sup>6</sup> C	7 N	°	9 F	10 Ne
<sup>11</sup> Na	<sup>12</sup> Mg											<sup>13</sup> AI	<sup>14</sup> Si	<sup>15</sup> <b>P</b>	<sup>16</sup> S	<sup>17</sup> CI	<sup>18</sup> <b>A</b>
<sup>19</sup> K	20 Ca	21 <b>Sc</b>	<sup>22</sup> <b>Ti</b>	<sup>23</sup> V	<sup>24</sup> Cr	<sup>25</sup> Mn	<sup>26</sup> Fe	27 Co	<sup>28</sup> Ni	29 Cu	30 Zn	31 Ga	32 Ge	33 <b>As</b>	34 Se	35 Br	36 <b>Kr</b>
37 <b>Rb</b>	38 Sr	<sup>39</sup> Y	40 <b>Zr</b>	41 Nb	42 <b>Mo</b>	43 <b>Tc</b>	44 Ru	<sup>45</sup> Rh	46 Pd	47 <b>Ag</b>	48 Cd	49 In	50 Sn	51 Sb	52 <b>Te</b>	53 	<sup>54</sup> Xe
55 Cs	56 <b>Ba</b>	Ln	72 Hf	<sup>73</sup> Та	<sup>74</sup> <b>W</b>	75 <b>Re</b>	76 <b>Os</b>	77 Ir	78 Pt	<sup>79</sup> Au	80 Hg	81 <b>Ti</b>	82 Pb	83 Bi	<sup>84</sup> Po	<sup>85</sup> At	86 Rn
87 Fr	<sup>88</sup> Ra	An	104 <b>Rf</b>	105 <b>Db</b>	106 <b>Sg</b>	<sup>107</sup> Bh	108 <b>Hs</b>	109 Mt	110 Uun		•		•	•	•		•

Lanthanides	57	58	<sup>59</sup>	<sup>60</sup>	61	62	63	64	65	66	67	<sup>68</sup>	<sup>69</sup>	70	<sup>71</sup>
	La	Ce	Pr	Nd	Pm	Sm	Eu	Gd	<b>Tb</b>	Dy	Ho	Er	Tm	Yb	Lu
Actinides	<sup>89</sup>	90	91	92	93	<sup>94</sup>	95	96	97	98	99	<sup>100</sup>	<sup>101</sup>	<sup>102</sup>	<sup>103</sup>
	Ac	Th	Pa	U	<b>Np</b>	Pu	<b>Am</b>	Cm	Bk	Cf	Es	Fm	Md	<b>No</b>	Lr

Fission products

Activation products

### **Used fuel characteristics (3/3)**

- Used fuel is initially highly radioactive due to short-lived fission products
- Radioactivity is a property of unstable isotopes which decay by giving off particles and rays (radiation) to become different (lighter) isotopes.
  - If the new isotope is also unstable, it will also decay
  - If the new isotope is stable, the process stops
- Every radioactive isotope has a "half life" which is the time until half of the atoms of the isotope decay
  - In 10 half lives, one thousandth (0.1%) of the isotope remains
  - In 20 half lives, one millionth remains
  - (Example: Carbon-14 dating measures the amount of <sup>14</sup>C left in plant and animal remains to determine when they died using a half live of 5,730 years)
- Most fission products have half lives of less than a second to a few days
  - Used fuel typically is "cooled" for at least 5 years to allow most of the fission products to decay away
  - The remaining fission products and transuranics have half lives that are measured in years to centuries or longer
  - The longer the half life, the longer the material remains radioactive, but at a lower level because less decay is occurring, and less radiation is produced

# **Used Fuel Disposition Options**

- After cooling, used fuel is currently stored waiting for final disposition
  - While used fuel becomes less radioactive with time, it remains a health hazard for thousands of years
- The disposition options are:
  - Direct disposal in a geologic repository designed to contain residual hazards for 100,000 years or more (current U.S. approach)
  - Recycling (practiced in some countries in Europe and Asia)
- Recycling separates the fuel:
  - Uranium and plutonium are recovered for reuse
  - Fission products and hardware are disposed in a geologic repository
  - Other transuranics may be recovered for reuse or included in the waste





### **Recycling Options**

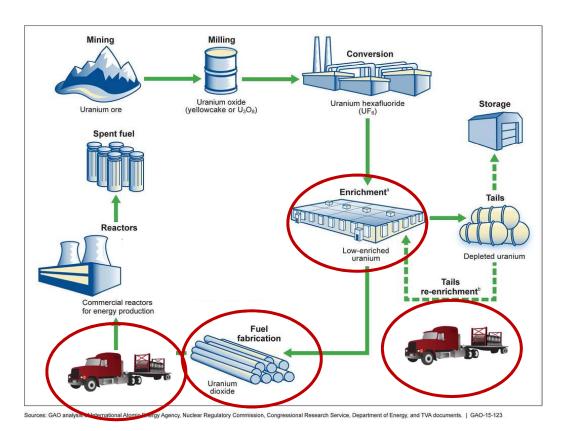
- With current reactors, only limited recycle is possible:
  - Recover the U for re-enrichment
  - Recover the Pu, which is mostly fissile
  - One recycle is feasible, after which too little fissile remains and the used fuel would be direct disposed
  - Results in ~30% more electricity from the original mined uranium and a small reduction in waste
  - Currently not cost effective
- With advanced reactors, continuous recycle may be cost effective:
  - Recover U, Pu, other transuranics (optional)
  - Irradiate in a fast spectrum reactor which supports enough breeding to produce fissile as fast as it is consumed
  - Recycle the resulting used fuel adding depleted uranium to make up for the fission products that are discarded
    - Existing inventories of depleted uranium would last for 3,000 years at the current level of nuclear generation without any new mining
  - Could result in over 100 times as much electricity from the original mined uranium and ~1/10<sup>th</sup> the waste to geologic disposal
  - Requires technology development

# **Commercial application of recycling abroad**

- France
  - UP-1 plant in Marcoule began operation in 1958 (~400 MT/yr)
  - UP-2 plant in La Hague began operation in 1967 (~400 MT/yr)
  - LWR oxide plant (UP2-400) began in La Hague in 1976 (400 MT/yr)
  - LWR oxide plant (UP3) began in La Hague in 1990 (800 MT/yr)
  - LWR oxide plant (UP2-800) upgrade in La Hague in 1994
- United Kingdom
  - Windscale plant for Magnox fuel began in 1964 (1200-1500 MT/yr)
  - THORP LWR oxide plant began in 1994 (~1200 MT/yr)
- Japan
  - Tokai-Mura plant began in 1975 (~200 MT/yr)
  - Rokkasho plant currently undergoing hot commissioning (800 MT/yr)
- Russia
  - Plant RT-1
  - Began operation in 1976, (400 MT capacity)
- China
  - Reprocessing pilot plant (60 MT/yr capacity)
  - Hot commissioning in progress
  - Planning 800 MT/year plant to begin operation in 2030

### Fuel Cycle Infrastructure Updates Needed to Support Advanced Reactors

- Enrichment
  - Variety of U-235 enrichments between 5 and 20 wt.%
- Fuel fabrication/de-conversion
  - Multiple fuel form options (metallic, oxide, liquid, etc.)
- Transportation
  - UF<sub>6</sub> to fuel fabrication facility
  - As fuel to reactor facility

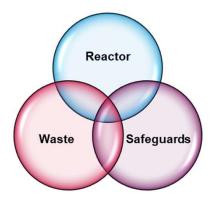


# **Fuel Cycle of the Future**

We don't know what it will look like, but we know what attributes are needed

- Cost competitive
- Manage proliferation risk
- Manage of waste
- Address safety and security







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